

SOTERIA PROJECT

SPEAKER/COORDINATOR: C. ROBERTSON (CEA)

Objectives

Expected Results

Starting point, context & approach

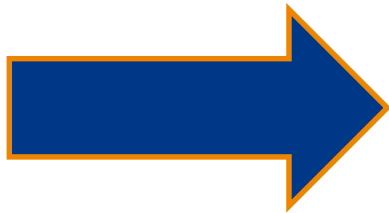
Work plan structure

Consortium

- The **nuclear energy safety issue** stands to the frontline of public debate



A priority for regulators and nuclear power providers to continue operating existing NPPs beyond the originally anticipated time frame by **extending their service life**



A need to guarantee a continuous safe long-term operation of existing power plants, it is a *sine qua non* to get an improved understanding of the role of **ageing phenomena** in reactor components

□ **Ageing management** key aspects:

- Identification of critical components subject to ageing (**internals and RPV steels**).
- Understanding the **main ageing mechanisms** affecting each critical component (**IASCC** and **fracture toughness** evolution)
- Use of operator's feedback towards improving the overall process

□ **From LONGLIFE** (mainly experimental project) :

- Microstructural and mechanical evolution data related to long-term irradiation exposure of RPV steels
- Experimental observation of:
 - **Flux effects** on radiation-induced microstructural features → ion/neutron irradiation experiment transferability issue?
 - **Data scattering** in surveillance programs → origin of outliers on trend curve derivation?



→ Understanding the observed phenomena:
game-changing modelling developments

□ From PERFORM60 :

- Development of **multi-scale modelling tools** to predict RPV and internals radiation-induced evolution
- Set-up of an **integrated platform** whose preliminary version was assessed by **end-users** from nuclear industry field
- BUT remaining questions relative to LTO and in-service inspection issues:
 - **Flux effects** on radiation-induced evolutions not addressed
 - **Bridging** between models developed at different scales **not complete**
 - Limited **comparison** of mechanical models with **experiments**
 - **IASCC** issue not dealt with respect to operating conditions (**chemical environment**, load transients, GB influence...)

The logo for the PERFORM 60 FP7 Project. It consists of the text "PERFORM 60" in a bold, blue, sans-serif font, with "FP7 Project" in a smaller, italicized, blue font below it. To the right of the text is a red and orange pixelated graphic.

→To set smart experiments at appropriate scales to calibrate, support and validate the proposed models

Improve the understanding and the prediction of **ageing phenomena** occurring in reactor pressure vessels and internal steels (internals)

Ensure a **safe long-term operation** of existing European nuclear power plants (NPPs)

Translate this newly obtained understanding in reliable **tools and methods** for industrial stakeholders

Provide **guidelines** for political stakeholders and future nuclear **safety** policies on national and European level

SOTERIA specific objectives



1

- Carry out experiments assessing neutron **flux and dose** effects on reactor pressure vessels and internal steels in pressurised water reactors

2

- Evaluate the residual lifetime of reactor pressure vessels by taking into account **metallurgical heterogeneities**

3

- Assess the effect of the **chemical environment** and dose on the integrity of internal structural components

4

- Develop models for the assessment of ageing mechanisms in RPV and internals and set of an **integrated computer-based platform** including the new modelling tools

5

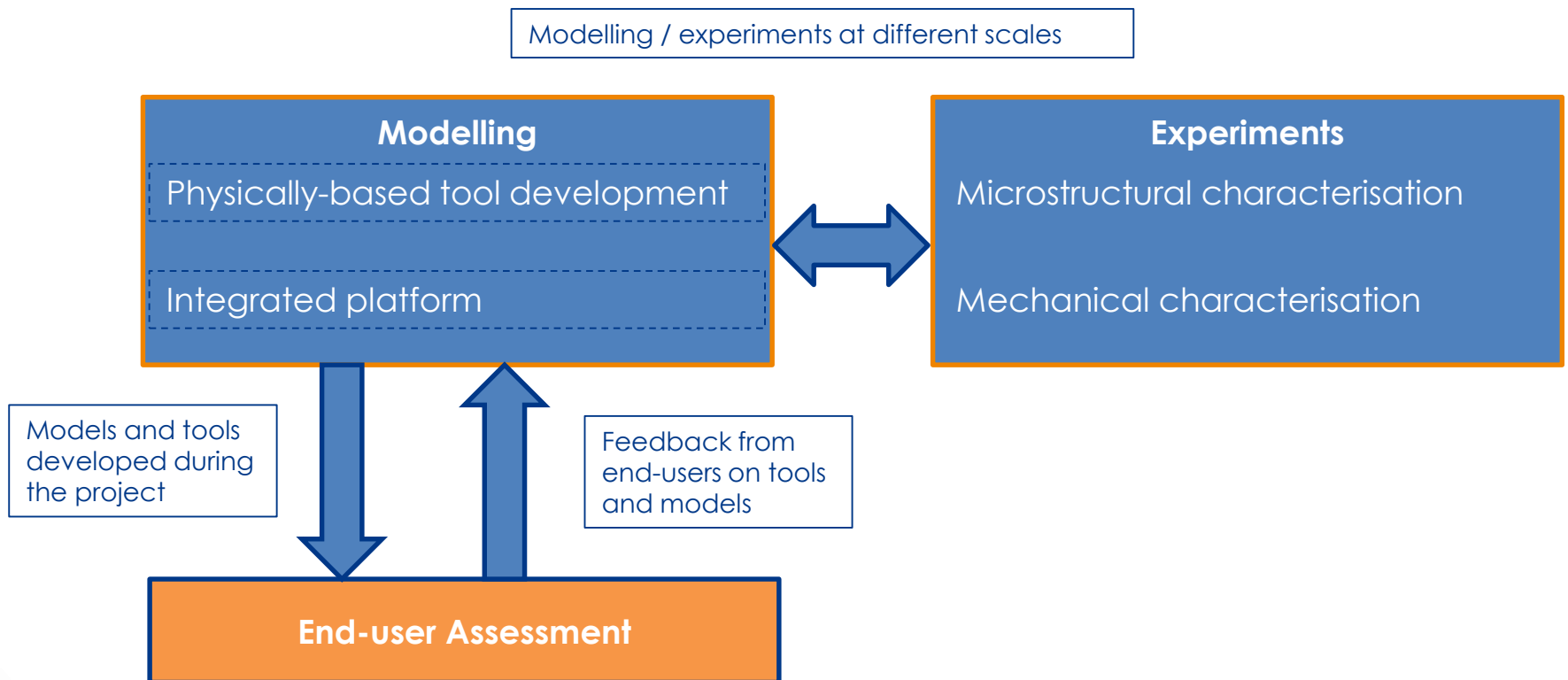
- **communicate** on the project achievements towards the nuclear engineering and research community in order to improve and harmonise the knowledge of ageing phenomena in nuclear power plants



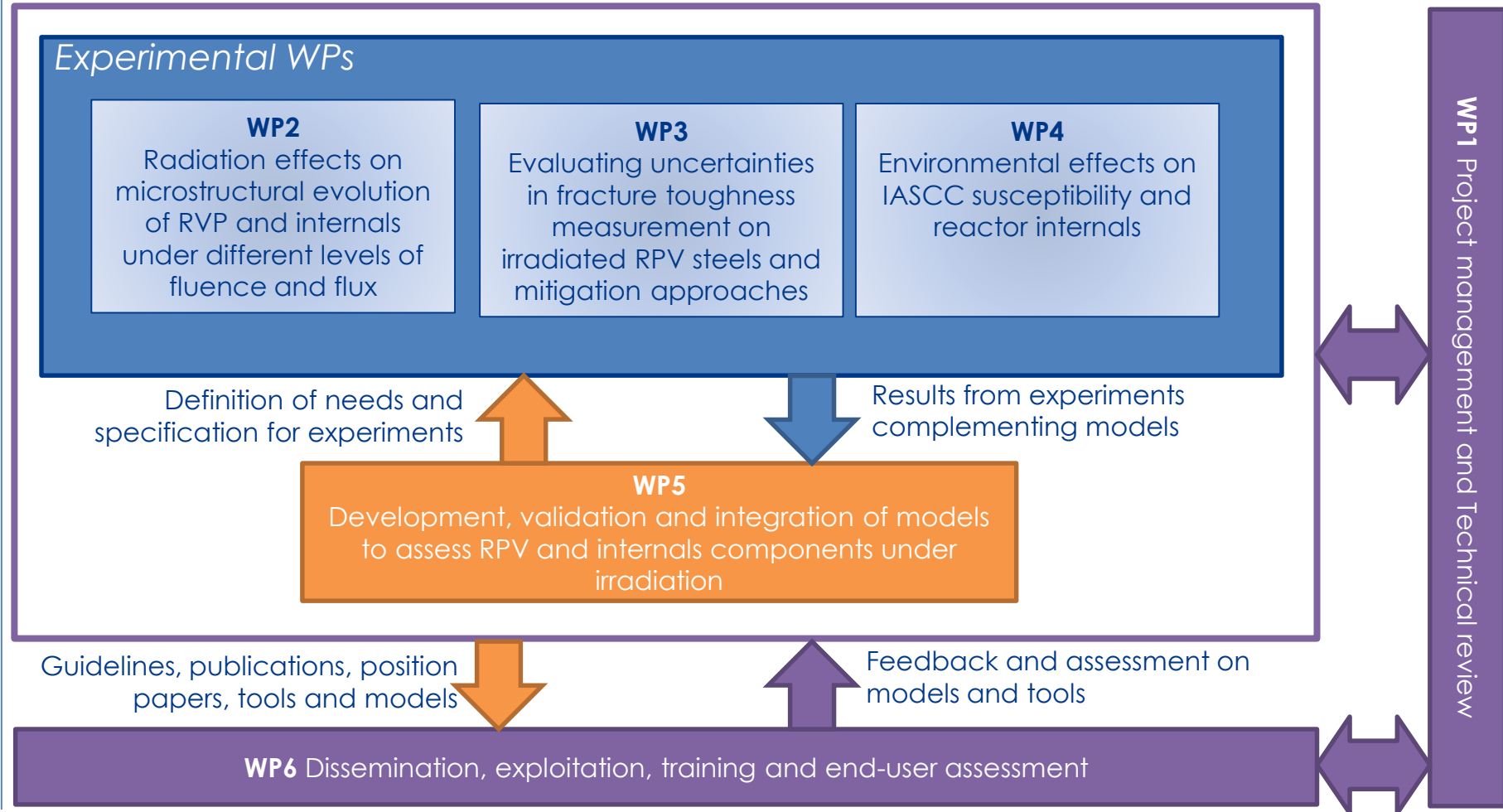
The SOTERIA approach (II/II)



- ❑ To start from an **end-user** perspective (end-user group has been arranged from the project start) to address operator-specific problems.
- ❑ Set up of **simulation-oriented experiments** aiming to validate models at different scales:



Workplan structure



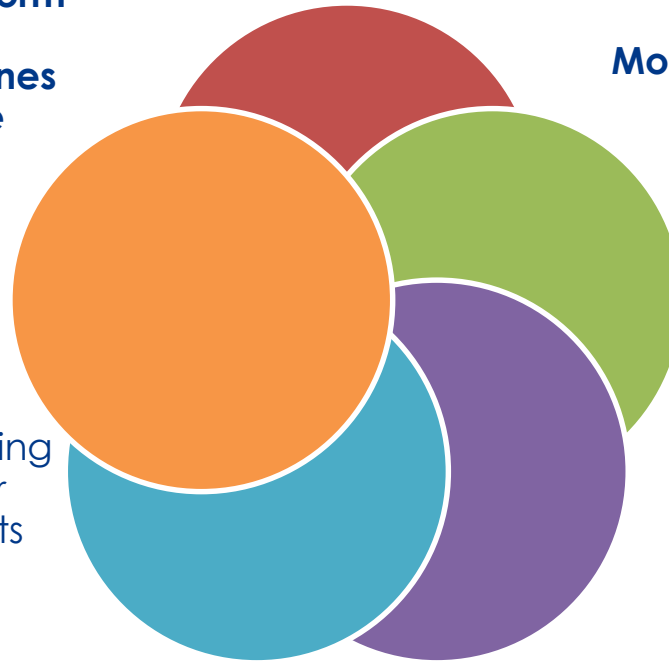
Understanding of initial microstructure heterogeneity effects on fracture

A specific **industry-adapted version of the modelling platform** for reactor safety margins evaluation, including **guidelines** and user-friendly **interface**

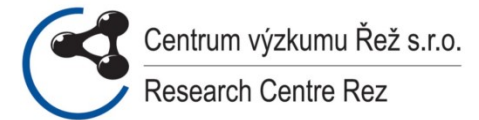
Models accounting for flux effect in support of ion/neutron emulation experiments

A modelling platform embedding **improved ageing models** for reactor structural components

A **database** collecting the experiments carried out in the project



- ➔ Final **User-Group** meeting
28th June, Tecnatom juline.vidal@edf.fr
- ➔ Newsletter final issue.



The SOTERIA Project Coordinator

Christian ROBERTSON
CEA
christian.robertson@cea.fr

The SOTERIA Project Office

Herman BERTRAND
ARTIC
bertrand@artic.eu

www.soteria-project.eu

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THANKS FOR YOUR ATTENTION

